



Ciernote Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

Insights on an SBO accident in BWR

Joan Fontanet, Eduard Diaz, Luis E. Herranz

Unit of Nuclear Safety Research CIEMAT



Unit of Nuclear Safety Research





y Tecnológicas

9th EMUG

CONTENTS

- Introduction Ι.
- *II.* Plant modelling
- *III.* **Results overview**
- IV. In-vessel insights
- Final remarks *V*.



Unit of Nuclear Safety Research



I. Introduction

• Framework

- Source term evaluation in a BWR SBO accident

Objectives

- To explore the role played by suppression pool nodalization in the containment thermal-hydraulics
- To find a conservative scenario for FP release and source term
- Scope
 - MELCOR v2.1 analysis of a BWR3-Mark1 plant

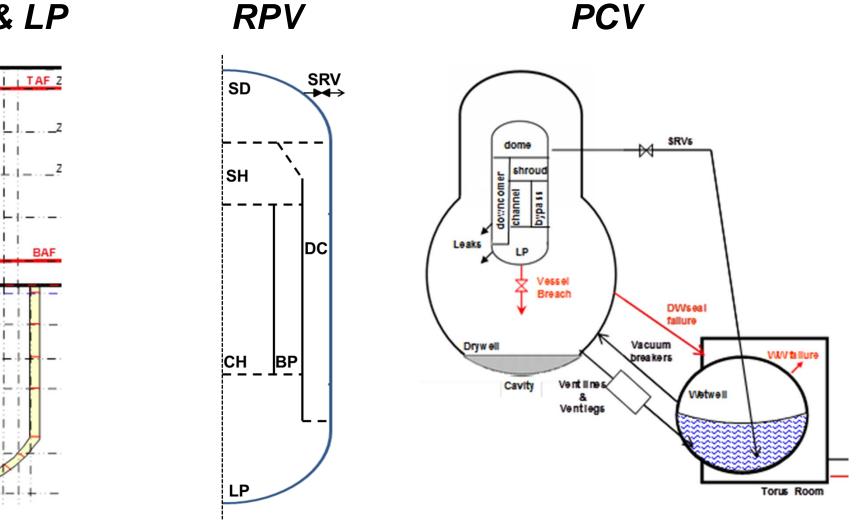




Ciernole Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

II. Plant modelling

Core & LP



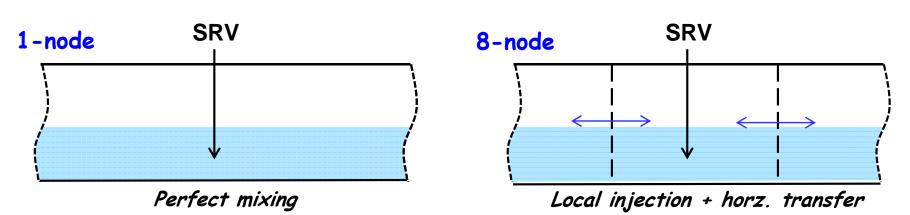


Unit of Nuclear Safety Research



II. Plant modelling (wetwell)

Azimuthal stratification

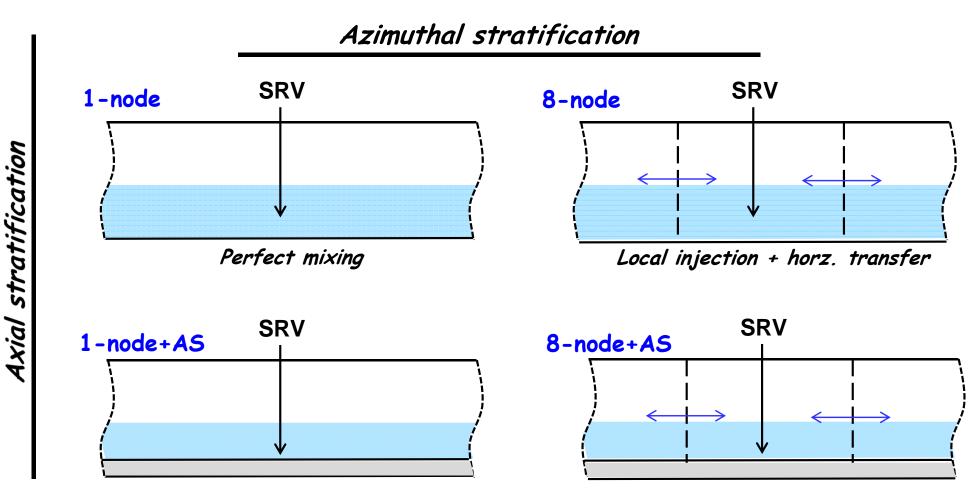




Unit of Nuclear Safety Research



II. Plant modelling (wetwell)



"perfect vertical stratification" (water below injection point does not play any role)





Y COMPETITIVIDAD

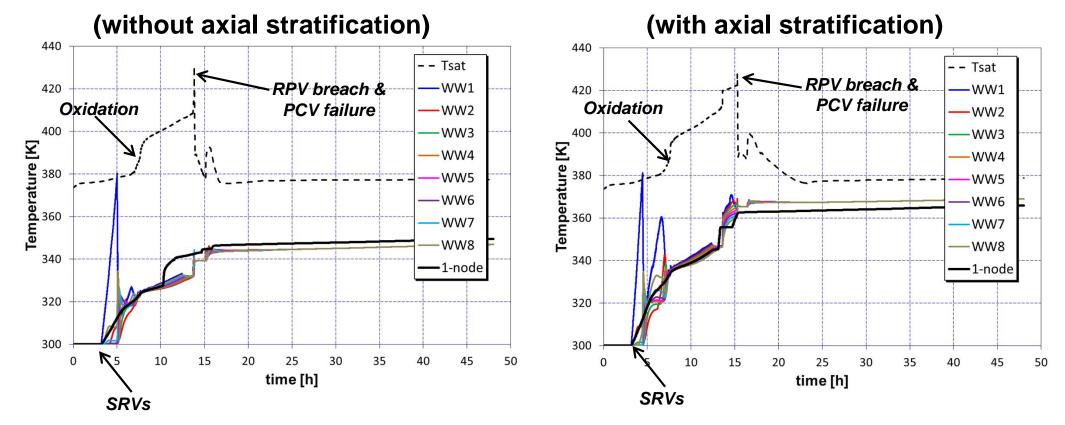
Ciemat DE ECONOMÍA, INDUSTRIA Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

9th EMUG

III. Results: Thermal-hydraulics

SP temperature

SP temperature





Unit of Nuclear Safety Research



Ciemat DE ECONOMÍA, INDUSTRIA Centro de Investigaciones Y COMPETITIVIDAD Energéticas, Medioambientales y Tecnológicas

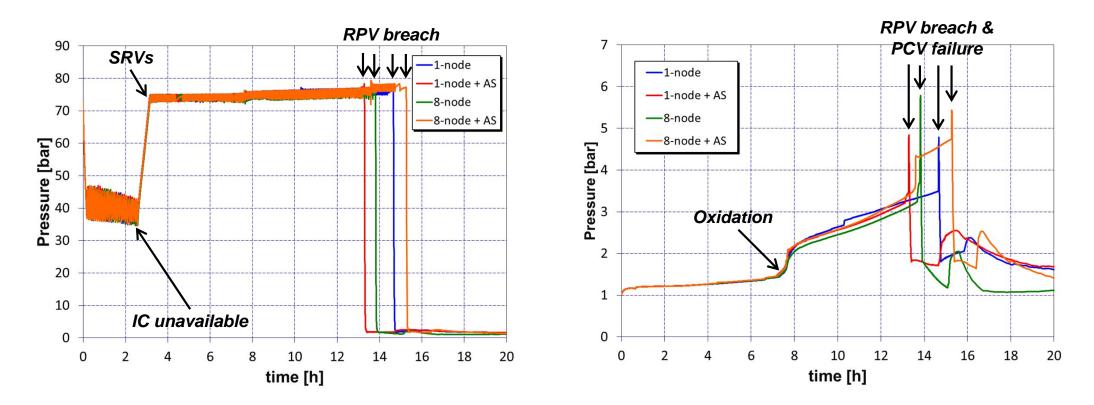
MINISTERIO

9th EMUG

III. Results: Thermal-hydraulics

RPV pressure

PCV pressure







III. Results: Key event chronology

Key event	1n	1n AS	8n	8n AS
IC operation (h)	0-2.6	0-2.6	0-2.5	0-2.6
SRV operation (h)	3.2-14.2	3.2-13.3	3.2-13.8	3.2-14.7
Core uncovery (h)	5.2-6.8	5.2-6.8	5.2-6.8	5.2-6.8
Fuel in LP (h)	10.3	13.1	13.6	13.5
Dry RPV (h)	11.9	13.3	13.8	14.7
RPV failure (h)	14.7	13.3	13.8	15.3
PCV failure (h)	14.7	13.3	13.8	15.3



Unit of Nuclear Safety Research



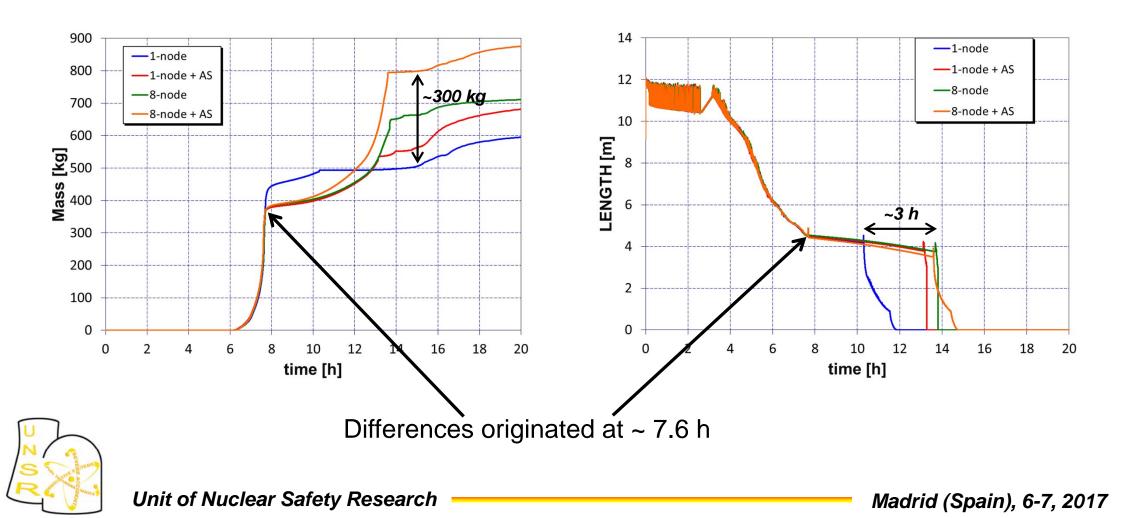
Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

9th EMUG

III. Results: In-vessel evolution

H₂ production

Water level



MINISTERIO DE ECONOMÍA, INDUSTRIA Y COMPETITIVIDAD

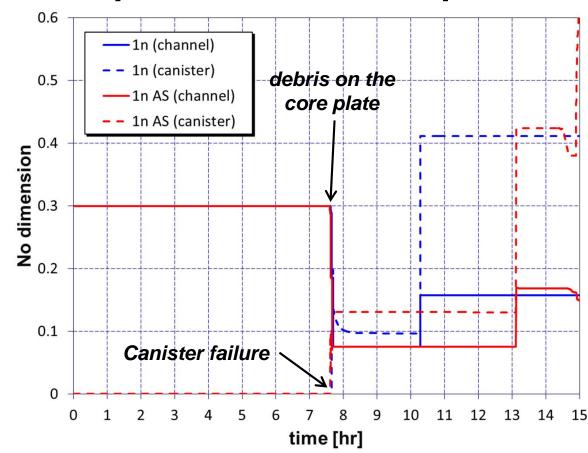
GOBIERNO

DE ESPAÑA

Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

IV. In-vessel insights

Open fraction of flow paths



Comparison of cases 1-node vs 1-node + AS

 Failure cross section of canister has a difference ~25%



Unit of Nuclear Safety Research

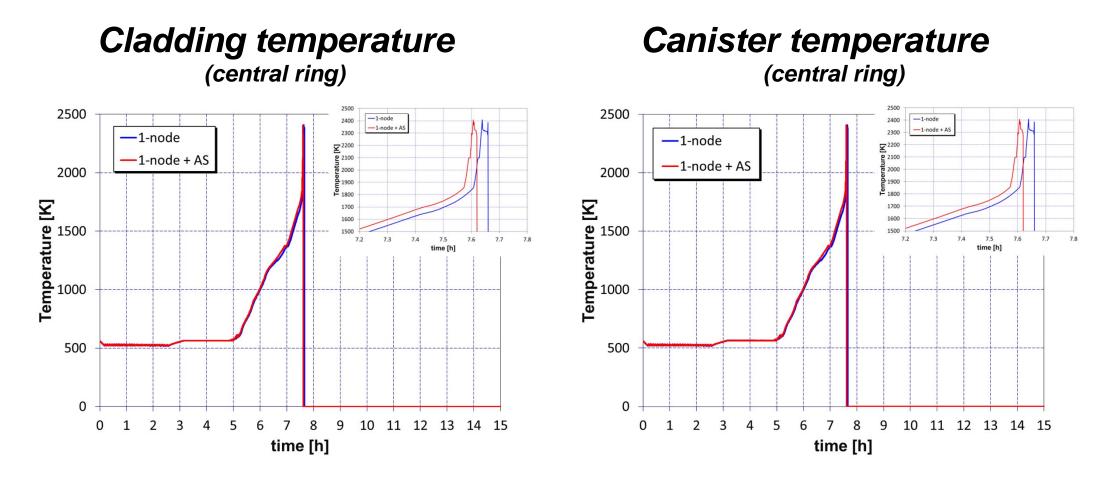


IV. In-vessel insights

Ciemot

Centro de Investigaciones

Energéticas, Medioambientales y Tecnológicas



Insignificant differences in clad and canister temperature (~5 K and ~ 3 min)

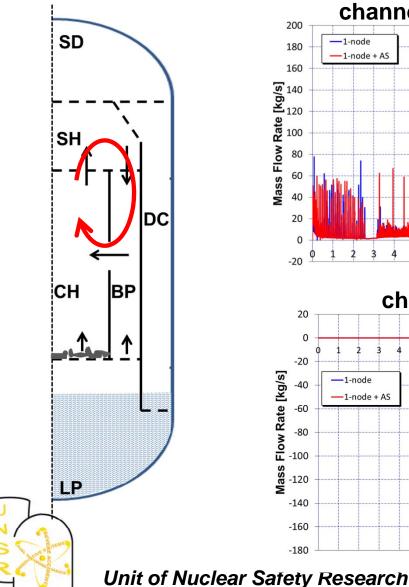


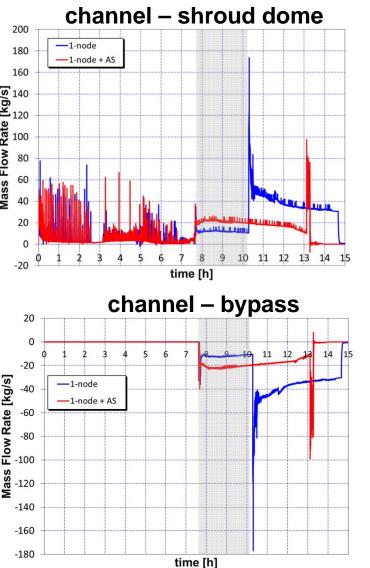
Unit of Nuclear Safety Research

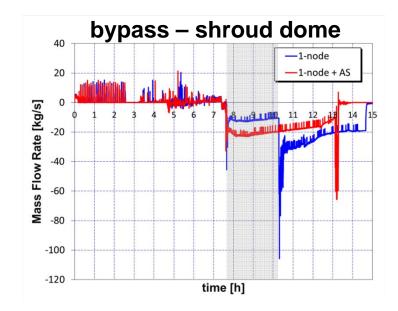
GOBIERNO MINISTERIO DE ESPAÑA DE ECONOMÍA, INDUSTRIA Y COMPETITIVIDAD Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

9th EMUG

IV. In-vessel insights







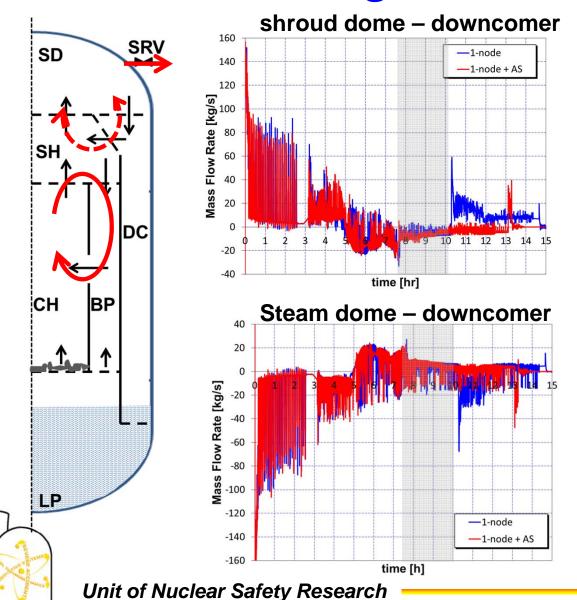
- A convective flow loop is developed in the core region when canister fails
- Case '1-node' has lower mass flow rate through the channel (lower CH-CP cross section)

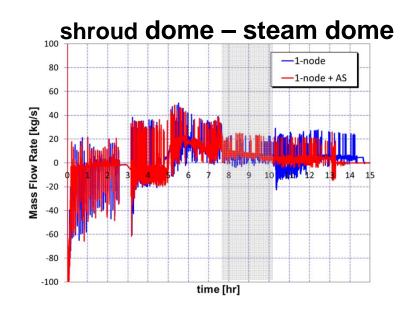
9th EMUG



MINISTERIO DE ECONOMÍA, INDUSTRIA Y COMPETITIVIDAD

IV. In-vessel insights





- A second flow loop is developed in the upper plenum
- This loop is broken when SRVs open
- Occasionally this loop can change the direction

GOBIERNO DE ESPAÑA

MINISTERIO

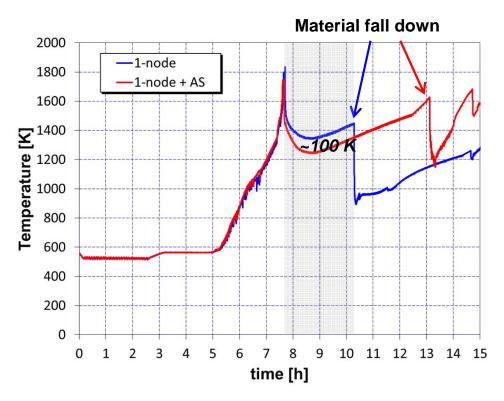
COMPETITIVIDAD

Ciemot DE ECONOMÍA, INDUSTRIA Centro de Investigacione Energéticas, Medioambientales y Tecnológicas

9th EMUG

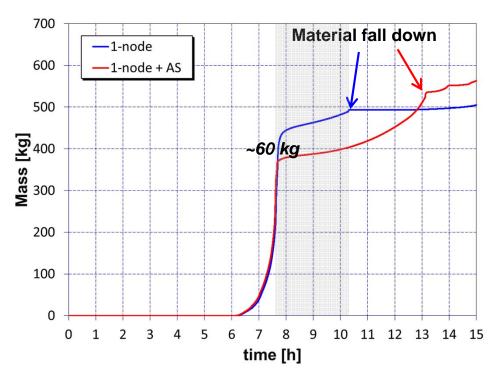
IV. In-vessel insights

Channel temperature



Lower flows leads to higher temperature

Hydrogen generation



Higher temper. leads to higher oxidation

Advanced degradation



Unit of Nuclear Safety Research



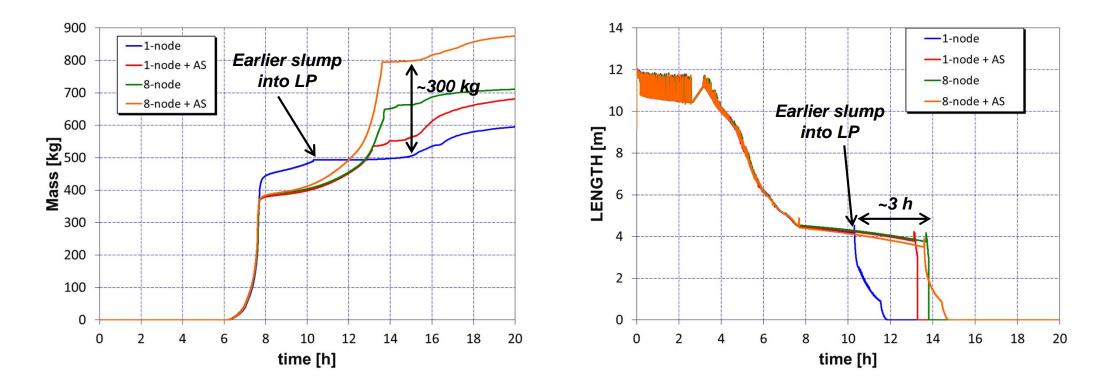
Y COMPETITIVIDAD

Ciemat DE ECONOMÍA, INDUSTRIA Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

IV. In-vessel insights

H₂ production





Advanced degradation, leads to lower in-vessel oxidation and earlier RPV dry-out

Unit of Nuclear Safety Research



V. Final remarks

- > RPV modeling introduces significant user effect on core degradation evolution.
- Canister failure leads to the development of convective loops inside the RPV.
- Small differences in estimated degradation can lead to significant differences in key parameters, e.g. generated H₂, PCV failure time, etc.
- Results must be taken with caution when supporting accident management.







Ciemple Centro de Investigaciones Energéticas, Medioambientales y Tecnológicas

9th EMUG

Acknowledgements

Spanish Nuclear Safety Council (CSN)

FASTNET project founded by EU



Unit of Nuclear Safety Research