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Overview of MELCOR Activities at KTH

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MELCOR Applications at KTH

- DBA safety analysis
 - Forsmark-1 2006 event
- > SA safety analysis for power uprate
 - Ringhals-3 PWR
 - Oskarshamn-3 BWR
 - Oskarshamn-2 BWR
- Analysis of in-vessel corium progression
 - To provide melt conditions for mechanistic study on in-vessel and exvessel coolability.
- Analysis of Fukushima accidents
 - F-I-3
 - F-I-2 see the seperate presentation of Chen et al
- ➤ SA analysis of spent fuel pools to be performed.



- Oskarshamn-2 is a BWR with the original capacity of 1800 MWth.
- The reactor is applying for 29% power uprate, to 2300 MWth operation..





SA Analysis of Oskarshamn-2

• Nodalization



CV160 Turbine Steam Lines CV190 CV180 Wetwell CV250 CV155 CV150 Feedwater CV215 CV140 CV135 -> Flow path CV175 CV110 CV176 (\rightarrow)

Containment





SA Analysis of Oskarshamn-2

• Nodalization



Core



• Some results of SBO



Pressure in the vessel



• Some results of SBO



H2 generation in the core



• Some results of SBO



Corium ejection



Motivation

he goal of SA research at KTH is to resolve two long-standing severe accident issues of Nordic BWRs:

- Ex-Vessel Corium Coolability (EVC)
- Steam Explosion Energetics (SEE)

The issues are closely related to

• In-Vessel Corium Coolability (IVC) --- Mechanistic modeling, e.g., PECM/ANSYS Mechanic coupled simulation, DECOSIM simulation.

which determines

• *Melt ejection conditions (timing, location, mass, composition, temperature, rate, ...)*





Analyais of In-vessel Corium Progression

• Reference plant: a BWR with the capacity of 3900 MWth.





Some results of SBO



Melt materials and mass in the lower head



Analyais of In-vessel Corium Progression

Some results of SBO



UO2 mass in the lower head

ZrO2 mass in the lower head



Analyais of In-vessel Corium Progression

Some results of SBO



Steel mass in the lower head

Zr mass in the lower head



Some results of SBO



FeOx mass in the lower head



- Severe accident analysis of Oskarshamn-2 BWR: Sensitivity and uncertainty study.
- Further investigation on in-vessel melt progression of the Reference BWR Plant.
- > All input decks for Swedish NPPs will be adapted to MELCOR 2.1.
- Apply SNAP to MELCOR simulation (model development and dynamic visualization of results).
- > MELCOR simulation for SFP.
- Analysis of Fukushima accidents: Input improvement and documentation for Unit-2 and Unit-3.
- Comparison of MELCOR with MAAP for severe accident simulations of Oskarshamn-3 BWR (*collaboration with OKG*).