

PAUL SCHERRER INSTITUT



Manuel A. Pouchon :: Laboratory Head :: Paul Scherrer Institut

# LNM: Labor für Nukleare Materialien

NES Information Event, 18.10.2016

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  - Background / Aspects of Nuclear Materials
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  - Cladding & Fuel (***Nuclear Fuels***)
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    - Embrittlement in Spallation Source
    - In-Situ Creep Metals / Ceramics

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# Mission of LNM



- **CENTRE OF EXCELLENCE:**

The LNM is the principal research unit and national centre of excellence in Switzerland in the domains of (irradiated) **materials behaviour** and **ageing** in **nuclear installations**.

- **ACADEMIC R&D & SCIENTIFIC SERVICE**

It provides material-related academic R&D contributions and scientific services to the **sustainability** of **current** and **future nuclear installations** for electricity & heat generation or waste reduction as well as to the performance of **nuclear research facilities**. A special emphasis is placed to the **safety & safe long-term operation** of the CH NPPs.



- **Material ageing** in the primary circuit and its impact on **integrity, safety & lifetime**
- **Performance** and **safety** of **LWR core materials** in **service** and storage
- **Radiation damage** in structural and core/target materials of advanced nuclear and accelerator systems.
- **Material irradiation program at SINQ** in co-operation with the Spallation Neutron Source Division.
- **Post-irradiation examinations** and **failure analysis** in close cooperation with the Hotlab Division AHL.

- **KNOWLEDGE MANAGEMENT:**

LNM is engaged in academic **teaching** and **education** as well as in **knowledge transfer** in its activity fields contributing to the education of the future nuclear specialists and preservation of expertise & excellence.



- **INDEPENDENT EXPETISE:**

Its **independent expertise** and **excellence** are always available to the **Swiss safety authority**, e.g. for expertise's and consulting (TSO), and for the **industry**, e.g., for material examinations and failure analysis.



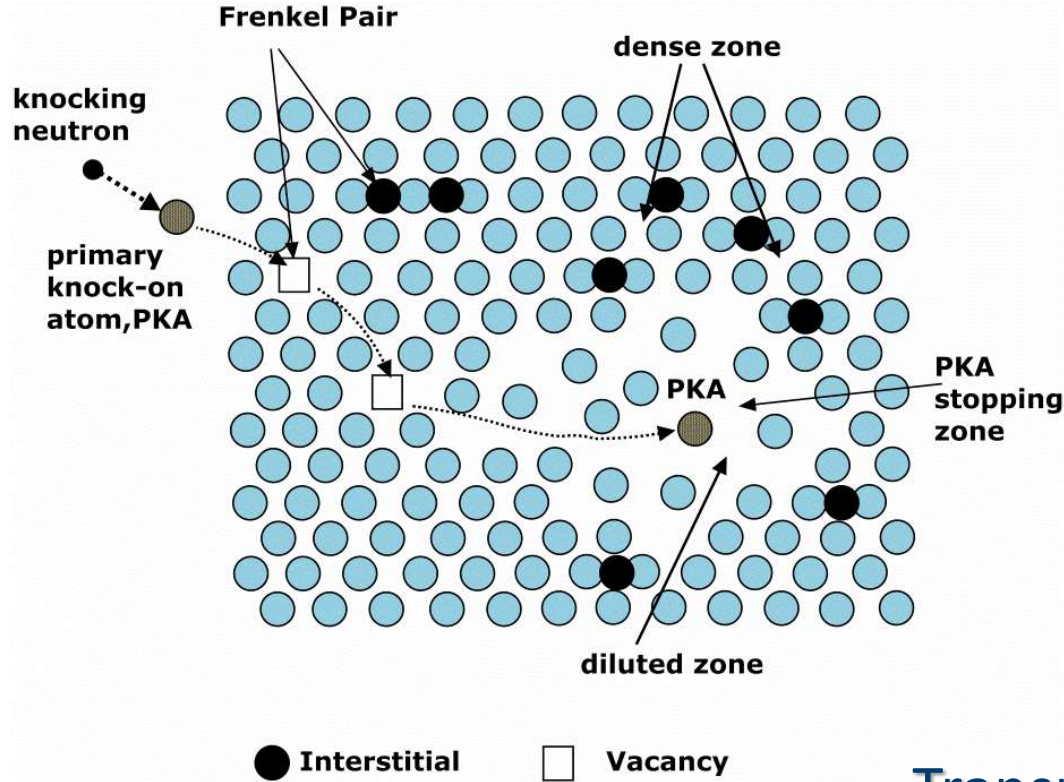
- **STATE OF THE ART INFRASTRUCTURE:**

It operates a state-of-the-art **lab & computing infrastructure** and **modelling tools** for the **characterization** of **(radioactive) materials** (strongly benefiting from PSI's unique large scale facilities: hotlab, SLS, SINQ, SwissFEL) and for the **analysis & prediction** of the **material behavior, integrity, safety & lifetime**.

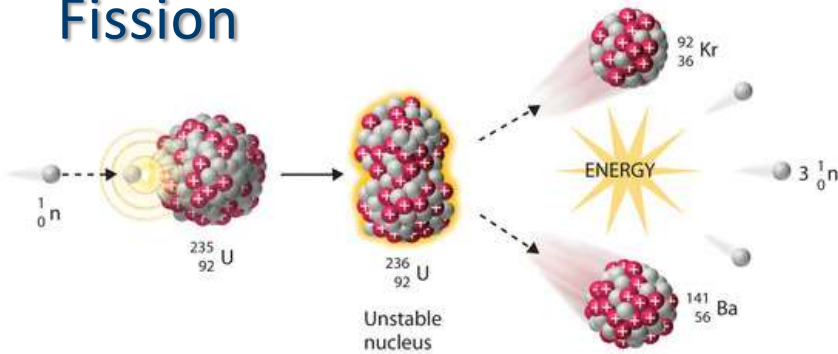


# Radiation Damage

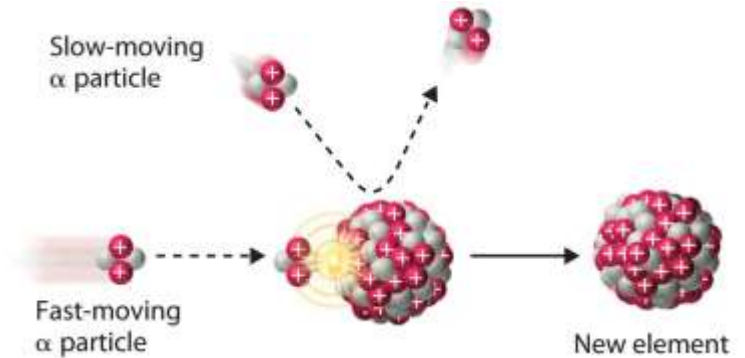
## Defects



## Fission

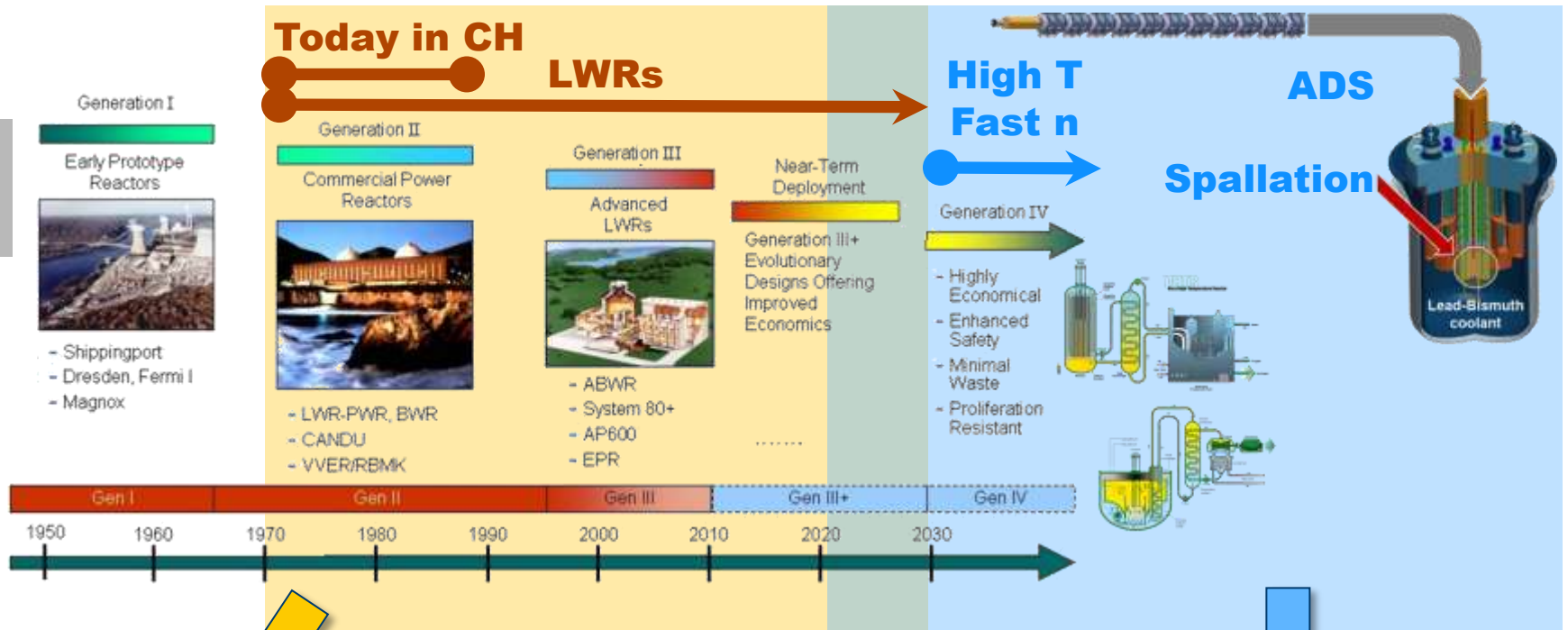


## Transmutation





# Background - Programs



**Nuclear Fuels**

**Component Structural Integrity**

**Advanced Nuclear Materials**

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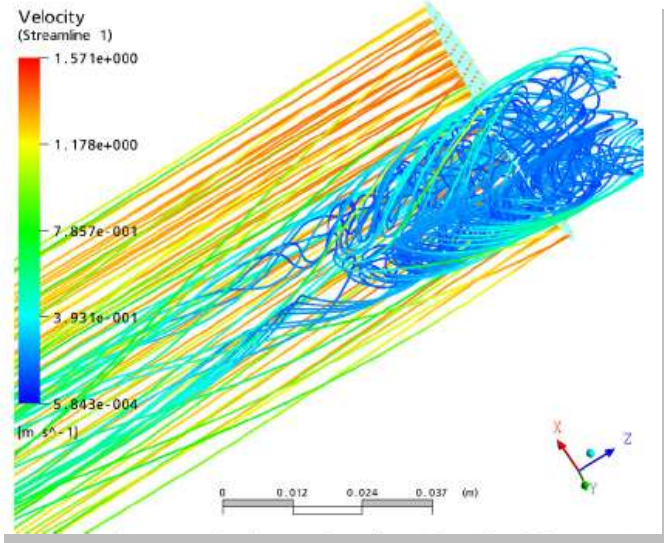
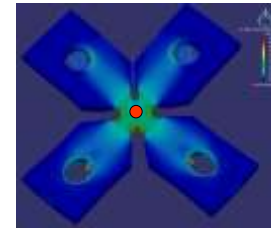
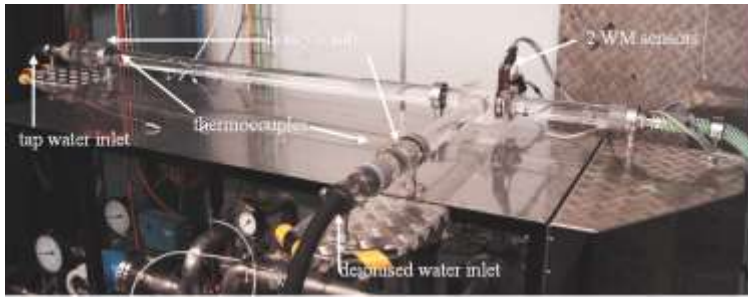


# TMF: PLiM-V & VI (swissnuclear, 2014-18)

Modeling & Lifetime Prediction of TMF Initiation & Growth under Turbulent Mixing Conditions in T-Joints & Experimental Validation

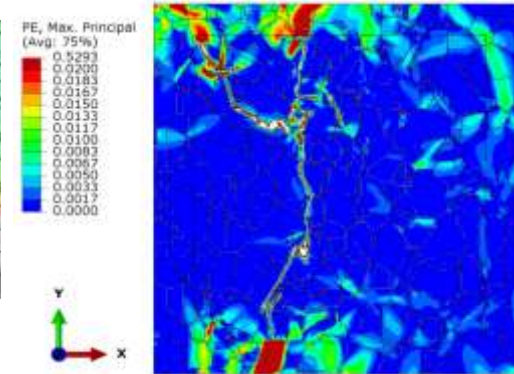
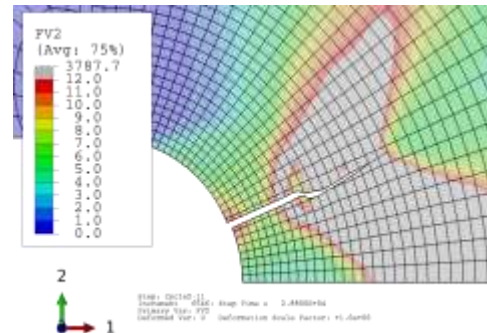
## CFD Modeling & Validation (by LTH)

## Structural Mechanics Modeling & Validation



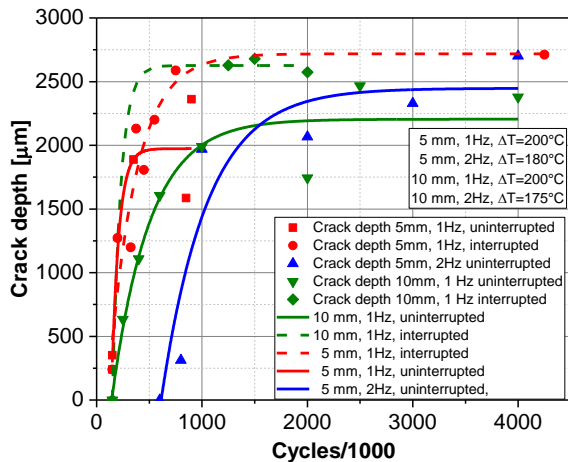
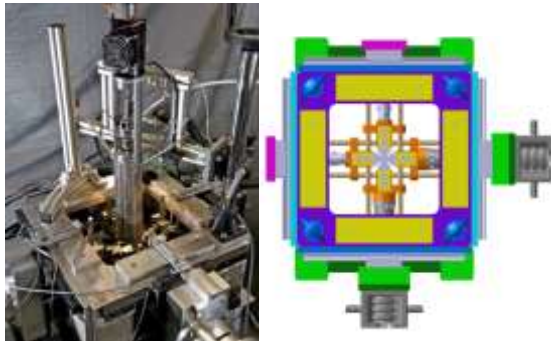
continuum-scale

meso-scale



# Non-Standard Fatigue Tests for Model Validation & Development

## TMF with biaxial pre-loading

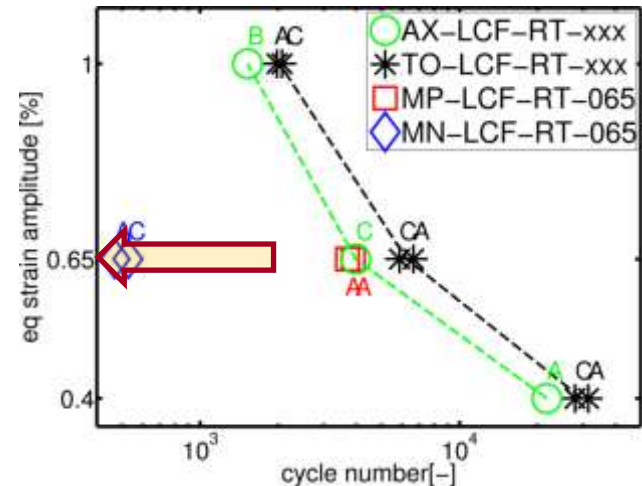
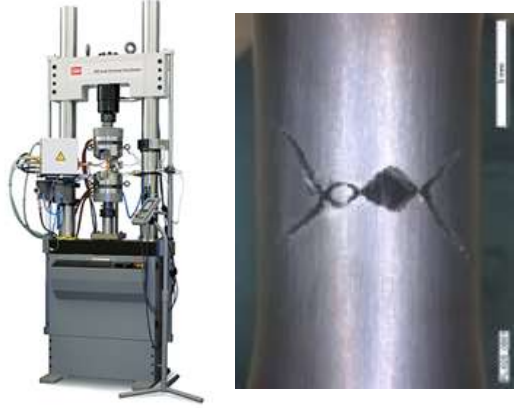


- Crack arrest at 2 to 2.5 mm = f (frequency)
- Strong effect of surface conditions on initiation

## HCF with micro notch (FIB)



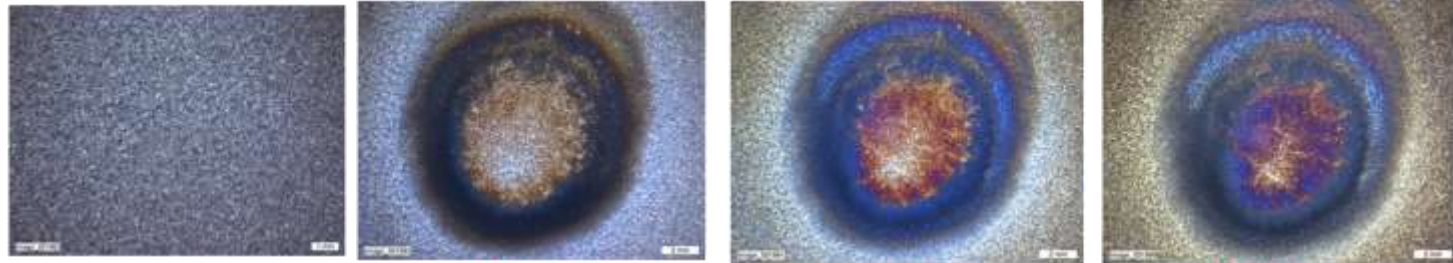
## Multiaxial HCF tension-torsion, proportional & non-proportional



Significantly shorter life in case of multiaxial non-proportional loading!

# Crack Network Formation due to Cyclic Thermal Shocks

Crack network formation due to cyclic thermal shocks ( $\Delta T=160\text{ }^\circ\text{C}$ , 1Hz, H<sub>2</sub>O)

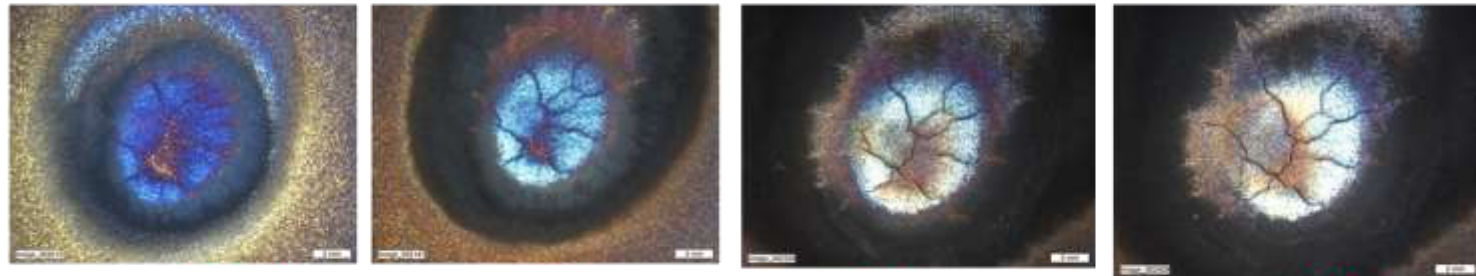


0 cycles

150 kc

400 kc

1050 kc



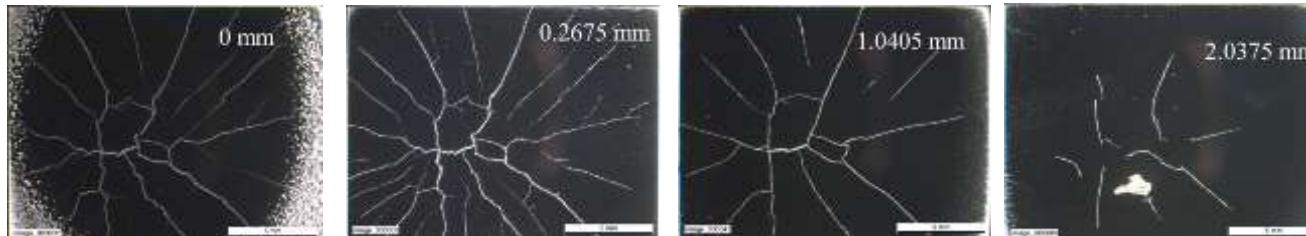
1950 kc

2450

3000 kc

3300 kc

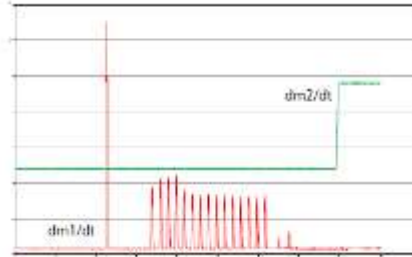
Crack network at different depths



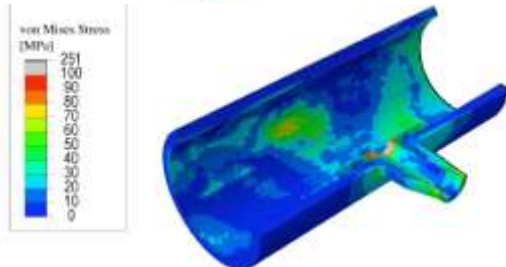
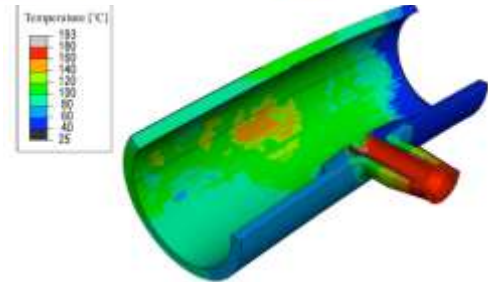
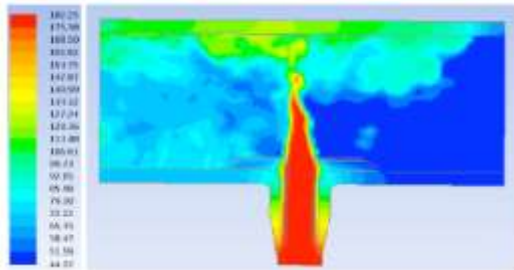


# Lifetime Assessment of Mixing T-Joint in KKM

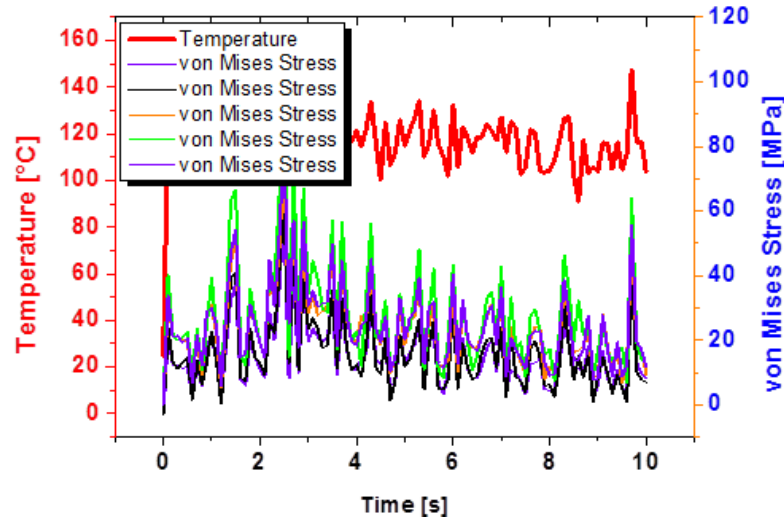
## Load Case 1



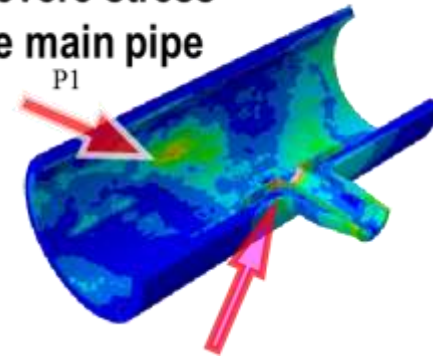
	Flow Rate [t/h]	Inlet Temperature [°C]	Coolant Pressure [bar]	Reynolds No.
Main	32	48	25	$7.4 \times 10^4$
Branch	12	180	25	$4.1 \times 10^5$



Stress & temperature at P1 vs time



Areas of most severe stress transients in the main pipe

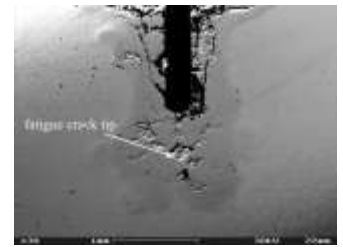


- Surface crazing not excluded for certain load cases
- Challenging evaluation of complex combined high & low frequency, multi-axial, variable amplitude fatigue load

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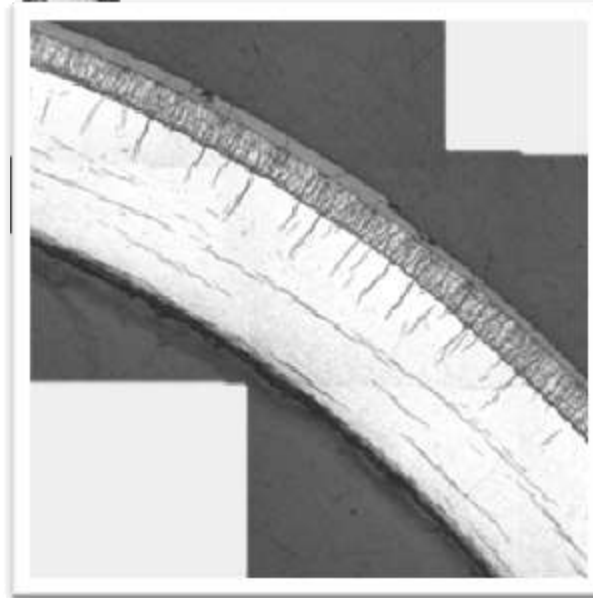
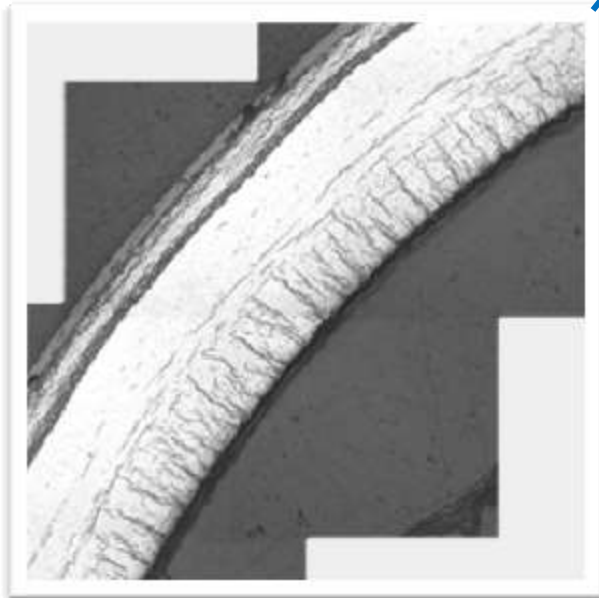
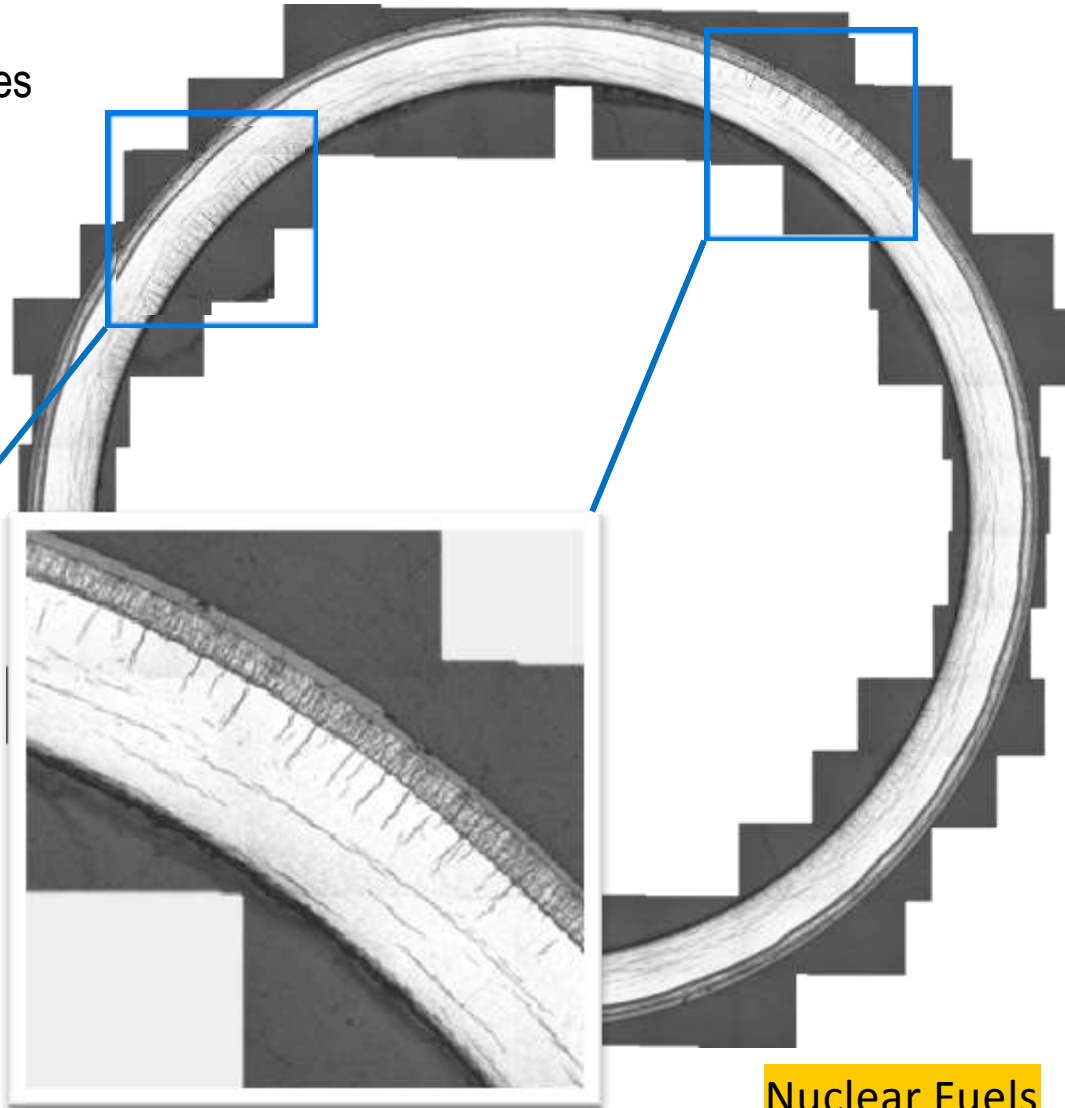
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# Hydrides in Cladding (Zircaloy)



## Hydrogen in the external liner:

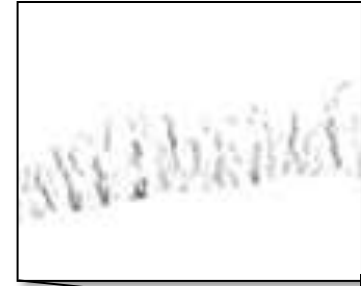
- hydrogen content in the liner varies with stress state;
- thorough inspection / relative quantitative estimation of hydrogen in the external liner will be necessary.



# Hydrides re-orientation KKG

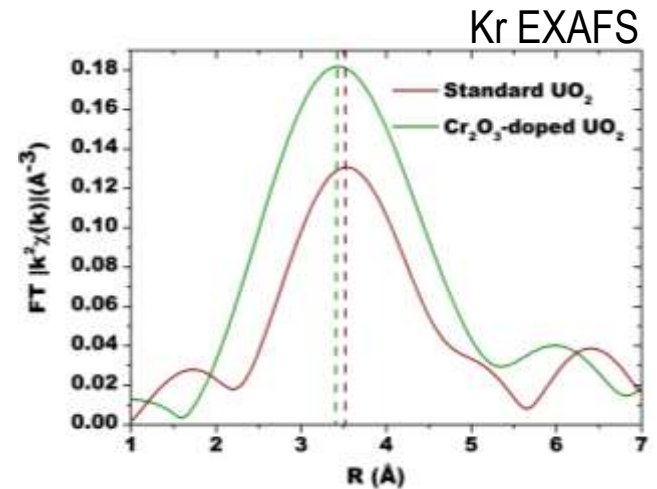
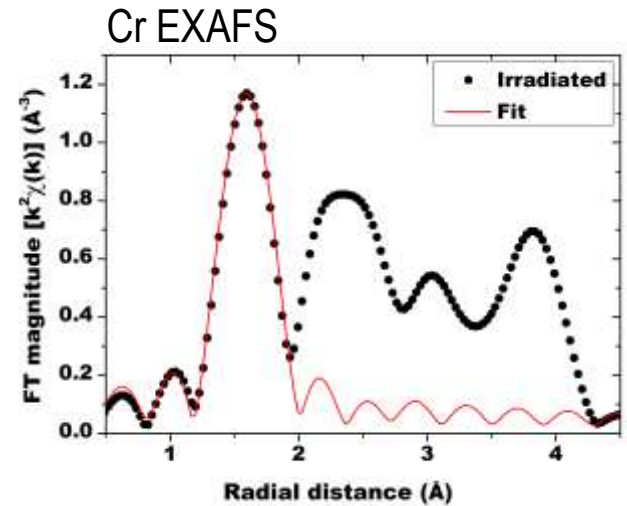
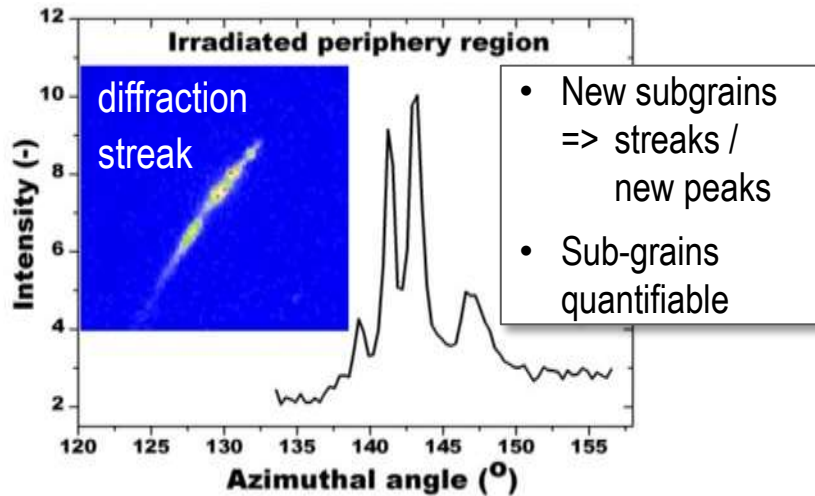
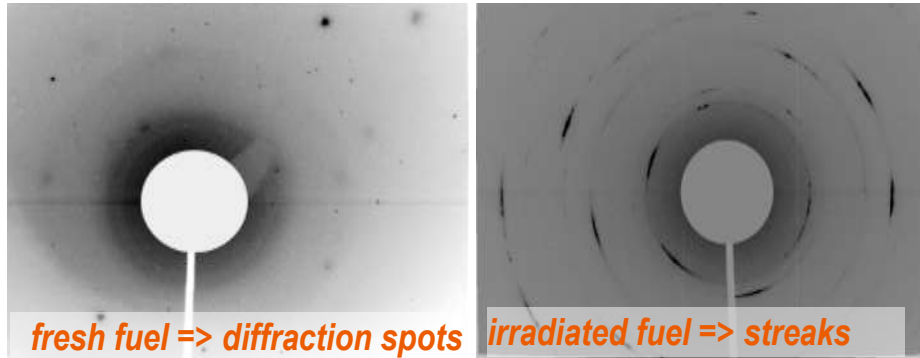
Zry-4 duplex DX-D4,  
AREVA, KKG  
BU 72 GW d t<sup>-1</sup>

**HYDIVA**





# Nuclear Fuel: XRD & EXAFS



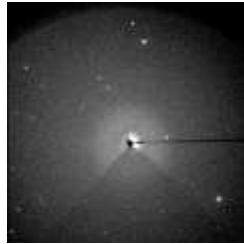
Work performed in multilateral collaboration

→ Hotlab (AHL)    → Industry (Areva)

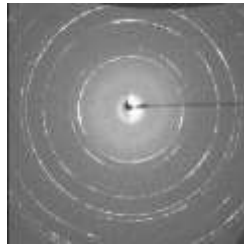
→ LNM    → Université Paris-Sud

**Nuclear Fuels**

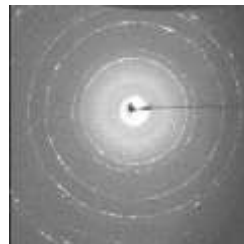
# High burn-up fuel & restructuring



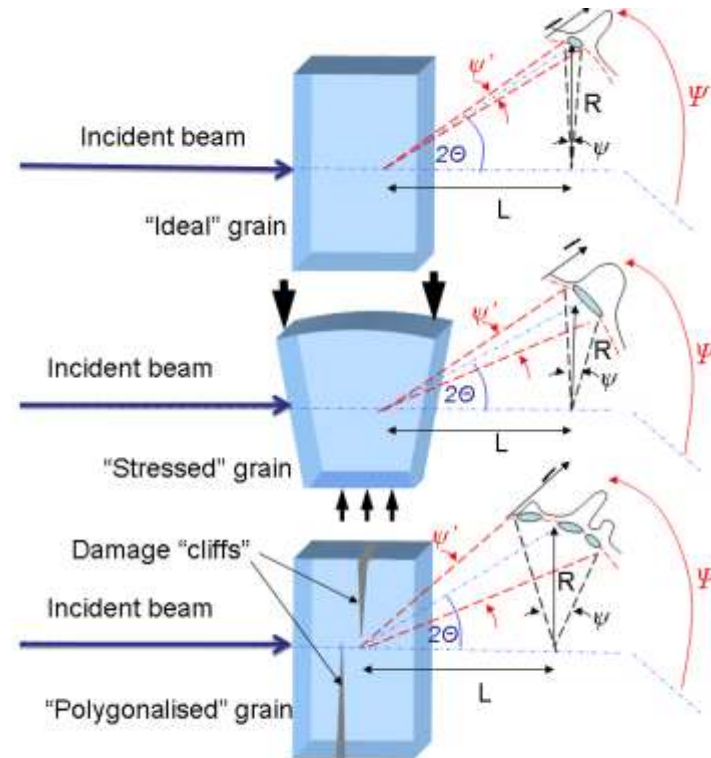
Fresh  $\text{UO}_2$   
Grain > beam size  
of  $1 \times 2 \mu\text{m}^2$



Pellet center  
(9 cycles)



Periphery  
(9 cycles)



## At 9 cycles:

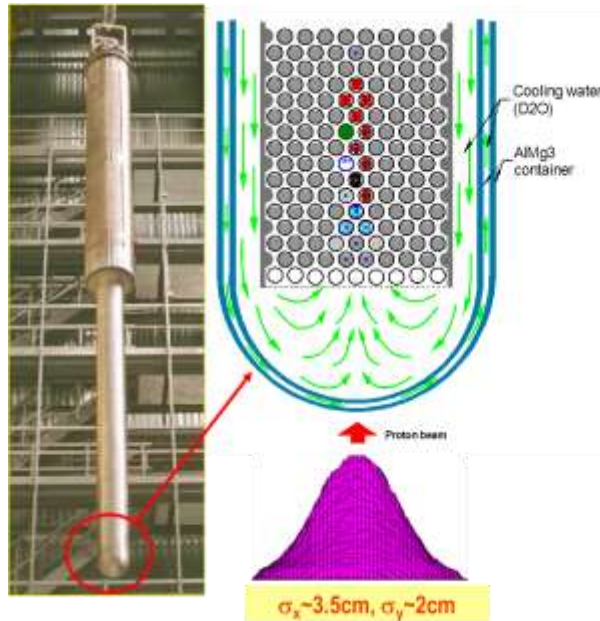
- quasi-full Debye rings (powder-like)
- very fine grains ( $< 1\text{-}2 \mu\text{m}$ ) in whole sample

## On-going analyses 5 and 7 cycles:

- polygonisation has started
- stresses?

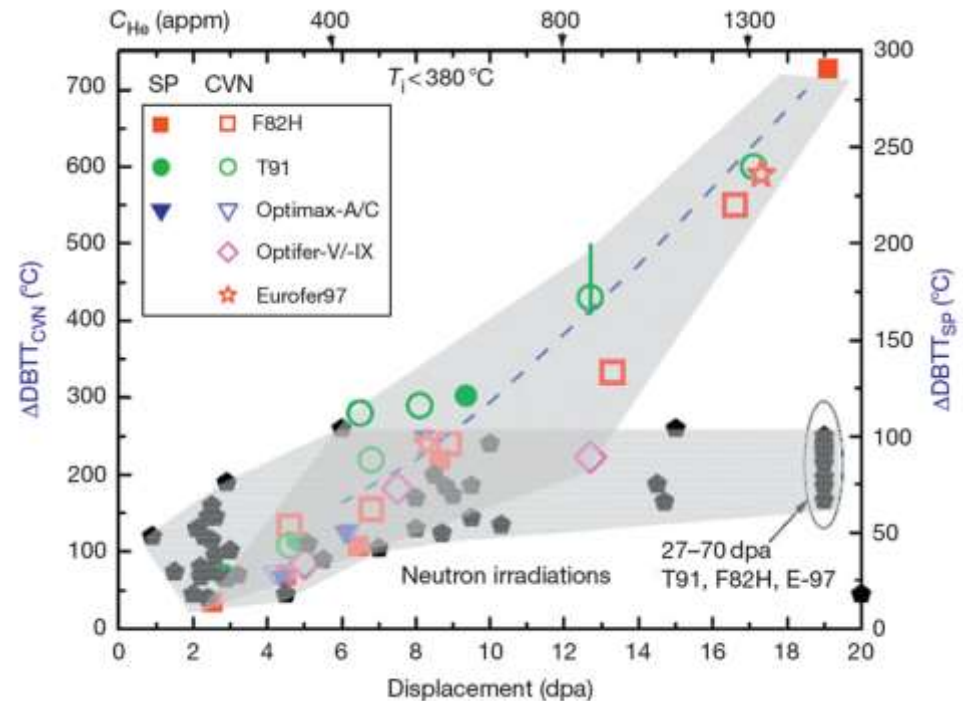
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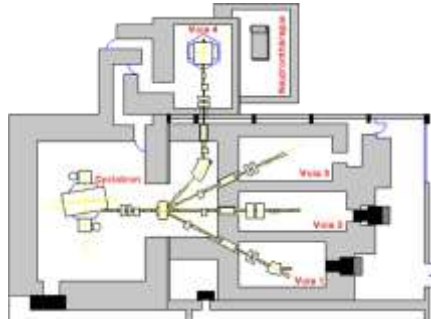


## Embrittlement of FM Steels

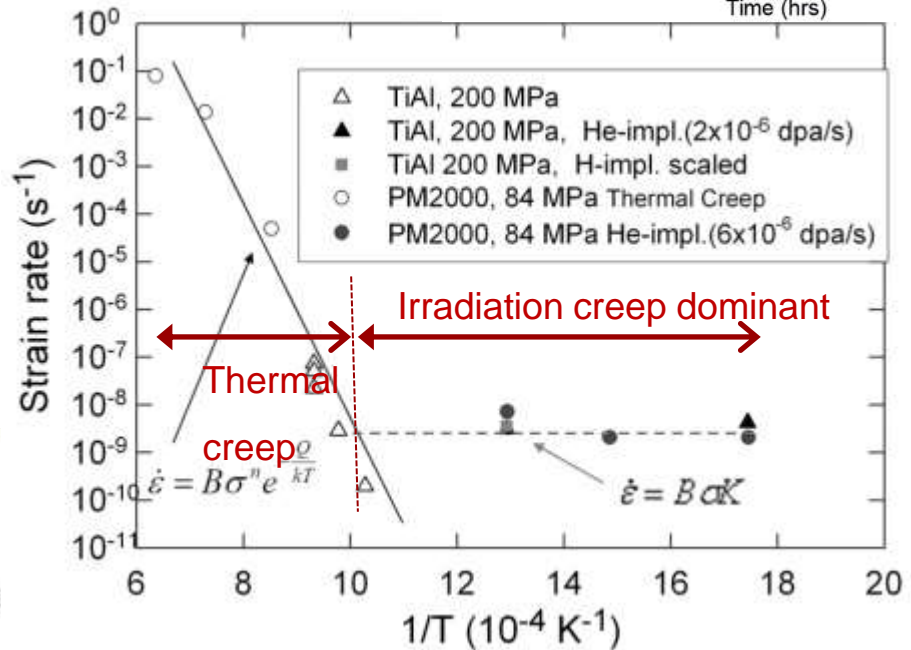
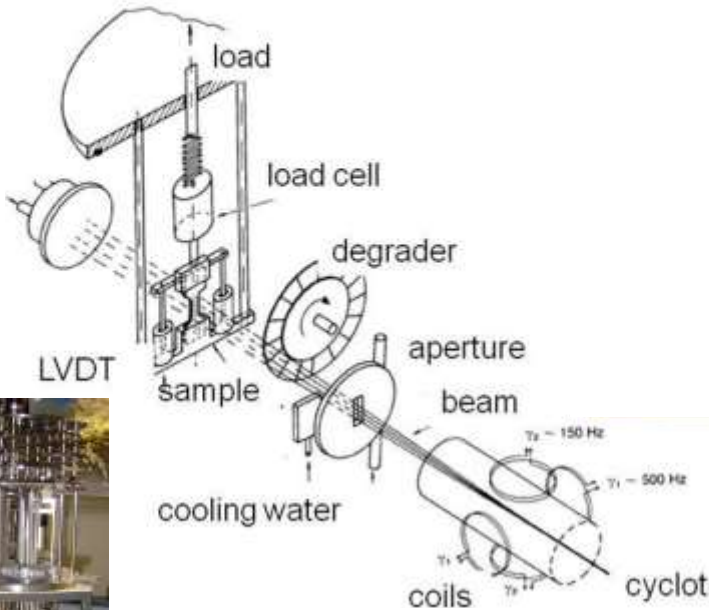
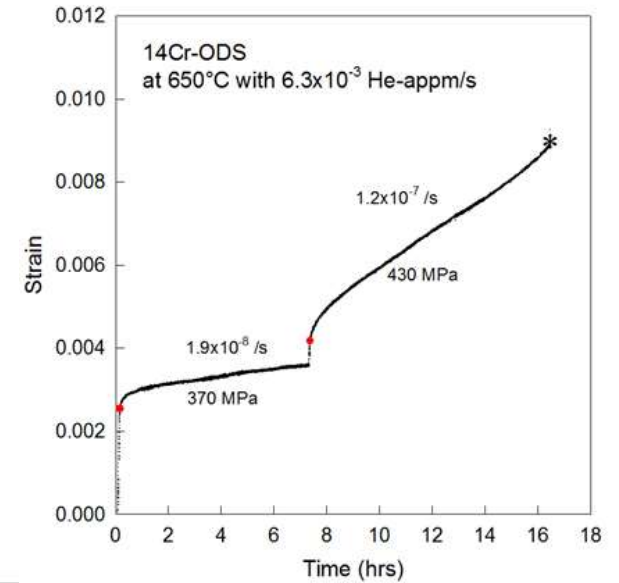
(high He production)



# In-Situ Irradiation Creep



Ions	Energy (MeV)	Intensity (μA)	Range (μm)
p	5--38	15	2800
H <sub>2</sub> <sup>+</sup>	5--25	13	380
d	5--25	40	760
α	10--50	15	380
<sup>3</sup> He	10--50	15	470

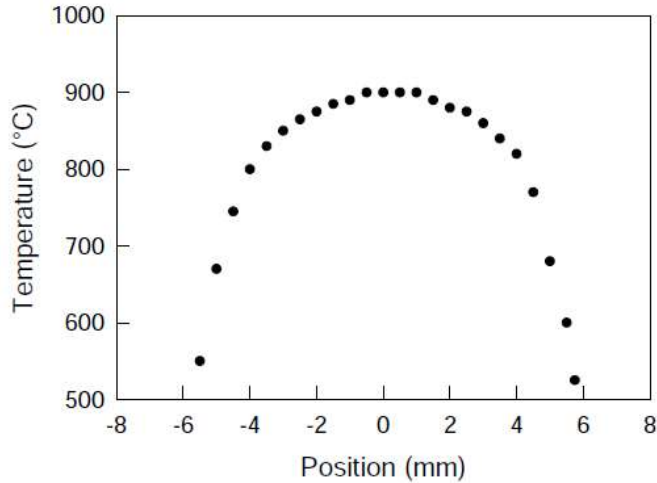




# Progress on Irradiation creep of SiC<sub>f</sub> composites

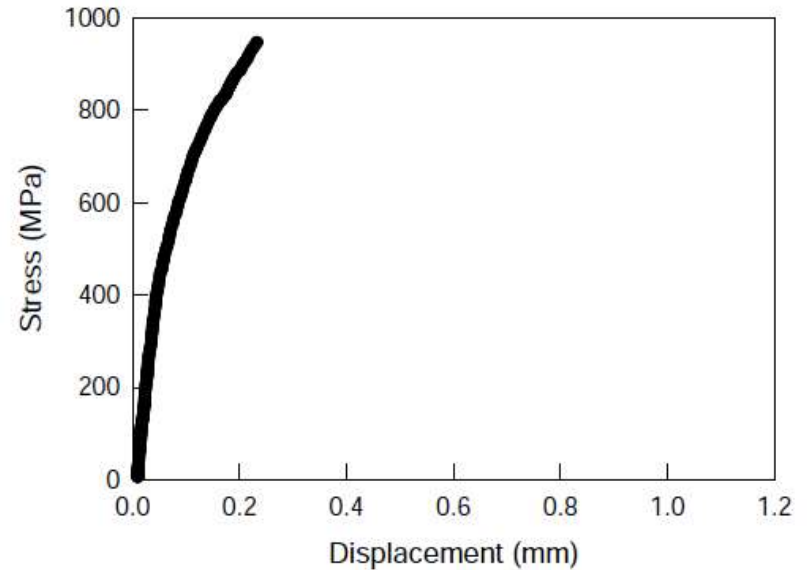
supported by French national program "Needs"

Temperature profile along the SiCf sample No.2

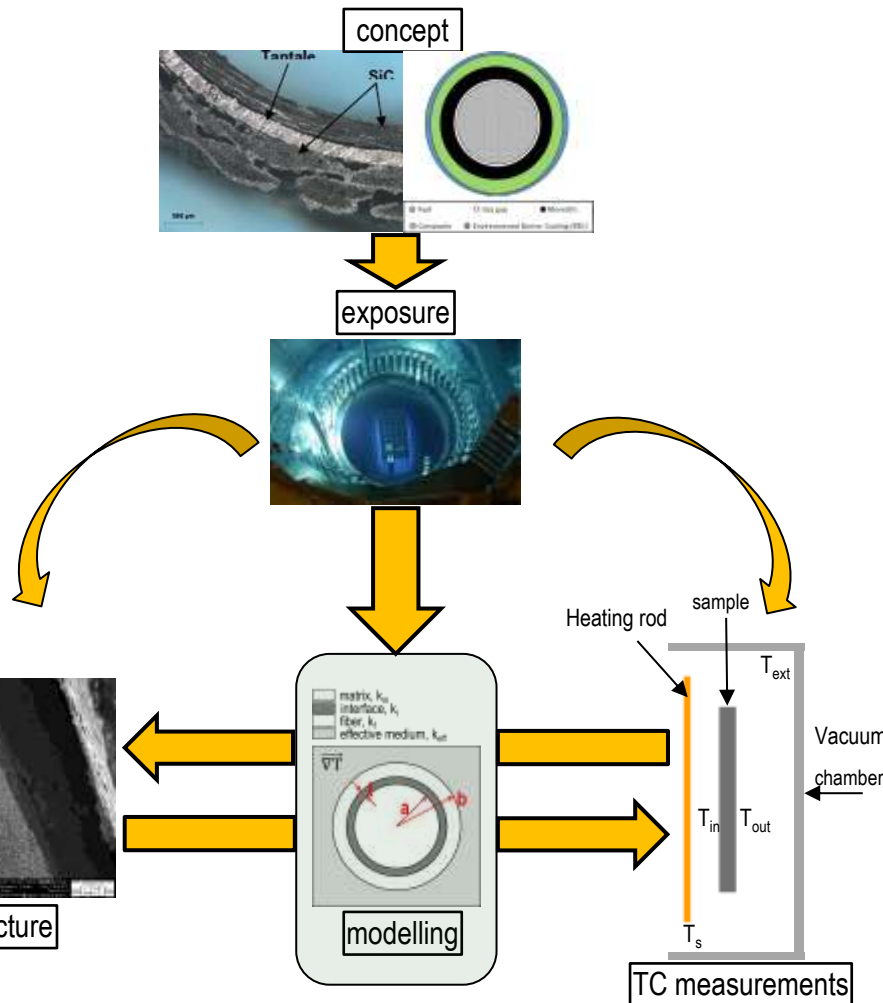


Aperture size: 12X4.5 mm<sup>2</sup>  
Beam current 5.6 μA

SiCf No. 2  
Tensile test at 900°C with beam



Uniaxial loading system for mini SiCf (0.5 mm in diameter) composites sample is successfully developed.



→ Effect/behavior of PyC interphase before/after irradiation

## Three blocks

- **TC measurements**  
on actual cladding tubes
  - Carried out on samples obtained through MatISSE and industry partners
- **Microstructure** study with EM
  - TEM → structure
  - EFTEM → effect of irradiation
  - FIB → samples and tomography
- **Modelling**  
link observations to measurements in a continuous medium based model



## **Nuclear Materials Research at PSI is active...**

... serving the authorities and industry with safety investigations.

... helping the industry with performance optimizations.

... contributing to education and training.

... contributing to the next generation research.





## My thanks go to

- All the members of LNM for their great efforts & performance
- AHL / SU / NES / NUM
- All external partners like ENSI, swissnuclear, CCEM, SNF, ...

