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Latest Experiences with MELCOR 2.2: An Overview of **Applications, Issues and Challenges**



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Areas of Work:



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Plants applications

1. BWRs: 1F1 Fukushima Analysis

2. iPWRs: SA scenarios assessment



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01

Scenario re-calibration in 1F1

• Frame

- OECD/NEA FACE project (2022-2026): Fukushima-Daiichi Nuclear Power Station Accident Information, Collection and Evaluation project.

• Objective

- Fukushima-Daichii accident analysis.

- Tool
 - MELCOR 2.2

• Scope

- Re-calibration of the possible scenario in Fukushima Daiichi Unit 1
 - (i) MELCOR version update (from MELCOR 2.2.9607 to r2024.0.3)
 - (ii) Scenario re-calibration

CODE EM (Evaluation Model)

CORE





SD

SH

LP

PCV

CAVITY





CAV 1

MELCOR version update





 ✓ Significant variation in accident progression among MELCOR versions.

Scenario re-calibration with r2024.0.3 (August 1, 2024) (Diagnosis based on P_{RPV}, P_{DW})

Hypotheses setting

- RCS leakages (GK & LP)
- No water injection before 24 h
- IC effect fitting
- SRVs (4;1 on)
- PCV venting
- Time of calculation = 24h

Scenario re-calibration



RCS leakages

- SRV gasket (steam) [10⁻²m 3·10⁻²m]
- "Solid" RCS (water) [0 − 5·10⁻³ m]

□ Thermal setpoints

• Penetrations failure [1153K – 1673K]

Materials interactions

- Eutectics formation
- Ad-hoc materials interaction





□ Main On-going work – Consolidation stage:

- $\,\circ\,\,$ Diagnosis based on $\mathsf{P}_{\mathsf{RPV}},\,\mathsf{P}_{\mathsf{DW}}$
- Postulated RCS leakages as an effective tool to guide the core degradation pattern

Generation Future work:

- Eutectic model fitting (delay in the liquefaction time)
- $\circ~$ Running time extension to 10 d $\,$
- \circ Explore other possible scenarios



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02

SA scenarios assessment in a generic iPWR

• Frame

The HORIZON 2020 SASPAM-SA project: aims to investigate the applicability and transfer of knowledge and know-how accumulated on the operating large-LWR to the iPWRs.
Coordinated by ENEA – 23 partners from 13 European countries

• Objective

- Evaluation of the code capabilities for a generic iPWR SA sequences.
- Explore the potential impact of different SSs availability on the accident progression.

• Tool

- MELCOR 2.2_18019 (SNL, UNRC).
- Original input from UNIROMA/ENEA.

Scope

- D2 (300 MWe, dry spherical containment, several passive SSs).
- Small LOCA (r = 2.14 cm) in one Direct Vessel Injection (DVI) line.



Plant Overview



- LWSMR
- 300 MWe iPWR design
- 17x17 XL fuel assembly W design
- Cylindrical RPV (6 m diameter x 21 m height)
- Spherical dry containment (25 m diameter)

• SPSs: LGMS, PSP, ADS, EHRS, EBT

SS	# Units	Actuation condition	Function	Connections (From/To)
EHRS	2	High DW P/Low PRZ P/Low PRZ level	Decay heat removal	Secondary coolant system/RWST
EBT	2	Low PRZ P	High pressure safety injection	Upper plenum/EBT tank EBT tank/DC
ADS st-1	3	Low PRZ P	Primary automatic depressurization	PRZ/QT
ADS st-2	2	Low LGMS tank mass	RPV pressure relief	PRZ/DW
LGMS	2	Low RPV/DW differential P	Low pressure safety injection	LGMS tank/DC



CODE EM (Evaluation Model)



RPV: 20 CVs Core: 6 radial rings, 16 axial nodes SSs: EHRS, PSS, LGMS, EBT, ADS Containment: 1 CV NPP: 97 CVs, 112 FLs, 121 HSs, 206 CFs No propriety data used



Scenario (DBA)

- Small LOCA in DVI line
- D2
- Steady state analysis carried out



CODE EM – Core nodalization refinement





Coarse nodalization



Detailed nodalization (based on SOARCA best practices)

Vessel Vol.	# CVs (coarse/detailed)	# FLs (coarse/detailed)
Core	1/20	1/35
Bypass	1/5	1/5
LP	1/1	1/5



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CODE EM – Core nodalization refinement



- Both calculations finish successfully(tend = $1.728 \cdot 10^5$ s)
- CPU time x 1.5 when detailed nodalization is used

• Both calculations fail (tend = $1.728 \cdot 10^5$ s)



CODE EM – Core nodalization refinement



Core damage slow down (10000 s in 65 %)

Minor changes:

- Containment failure delay (aprox. 1000 s) in the detailed calculation.
- H₂ production decrease ~ 12 % (detailed model)

Back to previous core modeling

Scenarios

SSs	DBA	SA1	SA2
ADS_st1	\checkmark	Х	Х
ADS_st2	\checkmark	✓	Х
EBT	\checkmark	✓	Х
LGMS	\checkmark	✓	Х
EHRS	✓	Х	Х
	DW Fast	DW	DW



Main results

Event time (s)	DBA	SA1	SA2
Start of the transient	0	0	0
Scram time	28	28	28
Actuation of the EHRS	38	-	-
Actuation of EBT	112	153	-
Actuation of ADS-1	128	-	-
Begin of core uncovery*	186	610	1690
Containment failure	-	4630	5730
Begin of core degradation	-	13450	10270
H ₂ production onset	-	12290	9070
Complete core uncovery**	-	23970	18680
Actuation of LGMS (s)	1770	44720	-
Actuation of ADS-2 (s)	11090	-	-
Time of vessel failure (s)	-	-	-

* Swell water level TAF reached

** Swell water level BAF reached





Main results



- Water reverse flow from the PSS to the DW.
- Attention to FPs redistribution.



Scenarios: SSs assessment

SSs	DBA	ADS_st1	EBT	LGMS	EHRS	ALL-EHRS
ADS_st1	✓	✓	Х	Х	Х	✓
ADS_st2	✓	Х	Х	Х	Х	✓
EBT	✓	Х	✓	Х	Х	✓
LGMS	✓	Х	Х	✓	Х	✓
EHRS	✓	х	Х	Х	✓	Х
LGMS EHRS	✓ ✓	X X	X X	✓ X	X ✓	× X



Conclusions/Remarks

- Passive SS performance as designed prevent any core damage in case of a small double-guillotine break in the DVI line.
 - Containment pressure was kept far from losing its integrity.
- □ SA scenarios might be possible if EHRS is not available.
 - The combined operation of all passive SSs except for EHRS is predicted not to be able to prevent core melting.
- □ Containment failure is unexpectedly estimated at early times.
 - This should be further explored to check its sensitivity to the scenario modelling.
- □ RPV integrity preservation, as predicted, should be confirmed with additional studies.
- □ Flow reversal by depressurization of the containment.
 - Particular attention should be paid in all regards: timing, intensity, consequences on FPs redistribution.
 - This should be proved not to be scenario model sensitive.

Conclusions/Remarks

□ In addition

- H₂ risk (AMHYCO project).
- Uncertainty quantification
 - Optimization of a BE case.
 - UQ of ex-vessel phase (in collaboration with UNIPI).
 - IUQ for SAs (in collaboration with UNIPI).

Looking ahead

- WC-ATFs.
- Non-WC.

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Thank you for your attention!

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